3rd ITER International Summer School: Plasma Surface Interaction Institut d'Etudes Politiques, AIX EN PROVENCE, 22-26 June 2009



Integrated numerical simulations and modelling of erosion and deposition on plasma facing walls

Kaoru Ohya

Institute of Technology and Science, The University of Tokushima, Japan

Contributions from K. Inai¹⁾, A. Kirschner²⁾, A. Ito³⁾, G. Kawamura³⁾, H. Nakamura³⁾, Y. Tomita³⁾ and T. Tanabe⁴⁾ ¹⁾Institute of Technology and Science, The University of Tokushima, Japan ²⁾Institut fuer Energieforschung-Plasmaphysik, Forschungszentrum Juelich, Germany ³⁾Department of Simulation Science, National Institute for Fusion Science, Japan ⁴⁾Interdisciplinary Graduate School of Engineering, Kyushu University, Japan

Outline

(A) INTRODUCTION TO EROSION/DEPOSITION

- a) Related issues to erosion/deposition
- b) Modelling of erosion/deposition
- c) Simulation codes for plasma wall interactions

(B) MODELLING AND INTEGRATED SIMULATIONS

- a) Projectile reflection and physical sputtering
- b) Chemical sputtering and hydrocarbon emission
- c) Impurity deposition and collisional mixing
- d) Thermal diffusion of impurities in materials
- e) Impurity transport in near-surface plasmas
- f) Molecular dynamics simulation of particle solid interactions
- g) Particle-in-cell simulation of plasma/sheath on surfaces

(A) INTRODUCTION TO EROSION/DEPOSITION

a) Related issues to erosion/deposition

- b) Modeling of erosion/deposition
- c) Simulation codes for plasma wall interactions

3rd ITER International Summer School: Plasma Surface Interaction, AIX EN PROVENCE, 22-26 June 2009 a) Related Issues to Erosion/Deposition in Fusion Devices

(1) Erosion of wall elements Reduced life time of wall elements

(2) Eroded impurities can penetrate into the plasmaDilution and radiation cooling of core plasma

(3) Redeposition of eroded impurities

Tritium retention in redeposited layers

Erosion, transport and redeposition of impurities is a crucial issue in fusion devices !

b) Modeling of Erosion/Deposition of Plasma Facing Walls

Carbon based materials for PFW



Erosion/deposition codes require to treat self-consistently:

I) Physical and chemical erosion *of* surface
II) Transport of released impurities *above* surface
III) Redeposition of returning impurities *on* surface
IV) Resultant material mixing *below* surface

Integrated simulation of erosion/deposition is NEEDED

c) Simulation Codes for Plasma Wall Interactions

(A) Ion-Solid Interactions —reflection, erosion, material mixing—

1) TRIM (static-MC, J.P.Biersack, L.G.Haggmark, Nucl.Instr.Meth. 174 (1980) 257) 2) ACAT (static-MC, Y.Yamamura, Y.Mizuno, IPPJ-AM-40 Nagoya Univ. (1985))

3) TRIDYN (dynamic-MC, W.Moller, W.Eckstein, Nucl.Instr.Meth.B 2 (1984) 814)
4) ACAT-DIFFUSE (dynamic-MC, Y.Yamamura, Nucl.Instr.Meth. B 28 (1987) 17)
5) EDDY (dynamic- and static-MC, K.Ohya et al., Jpn.J.Appl.Phys. 35 (1996) 4523; Rad. Eff Def. Sol.142 (1997) 401)

6) HCParcas (classical-MD, E.Salonen, K.Nordlund et al., Europhys.Lett. 52 (2000) 504)

7) MolDyn (classical-MD, D.A.Alman, D.N.Ruzic, J. Nucl.Mater. 313-316 (2003) 182)

- 8) Classical-MD : J.Marian e.al., J.Appl. Phys. 101(2007) 044506
- 9) Classical-MD : P.S. Krstic et al., New J.Phys., 9 (2007) 209.

10) Classical-MD : K.Inai, Y.Kikuhara, K.Ohya, Surf. Coat. Technol., 22-23 (2008) 5374.

11) Classical-MD : Z. Yang et al., J. Nucl. Mater. 390-391(2009)136.

(B) local deposition —Impurity transport in near-surface plasmas—

 WBC (3D-MC, J.N.Brooks, Phys.Fluids B 2 (1990) 1858)
 ERO (2/3D-MC, D.Naujoks et al., Nucl. Fusion 33 (1993) 581; U.Koegler et al., Report Jul-3361, juelich, 1997; A.Kirschner et al., Nucl. Fusion 40 (2000) 1421)
 DIVIMP (2D-MC, P.C.Stangeby et al., J.Nucl.Mater. 196-198 (1992) 258)
 EDDY (3D-MC, J.Kawata, K. Ohya, Jpn.J.Appl.Phys. 34 (1995) 6237)

(B) MODELLING AND INTEGRATED SIMULATION

- a) Projectile reflection and physical sputtering
- b) Chemical sputtering and hydrocarbon emission
- c) Impurity deposition and collisional mixing
- d) Thermal diffusion of impurities in materials
- e) Impurity transport in near-surface plasmas
- f) Molecular dynamics simulation of particle solid interactions
- g) Particle-in-cell simulation of plasma/sheath on surfaces

a) Projectile Reflection and Physical Sputtering



b) Chemical Sputtering and hydrocarbon emission

Hydrogen ion penetrates into carbon and forms hydrocarbon after thermalization, which diffuses to surface and desorbs.



Formalization by Roth [JNM266-269(199)51] :

$$Y_{chem}(E, T, \varphi) = \frac{Y_{low}(E, T)}{1 + \left(\frac{\phi}{6 \times 10^{21}}\right)^{0.54}}$$
$$Y_{low} = Y_{therm}(1 + DY_{dam}) + Y_{surf}$$

Y_{therm}: chemical erosion by thermalized ions
Y_{dam}: enhancement of thermal erosion by radiation damage
Y_{surf}: ion induced desorption of hydrocarbon radicals

Sputtering yield strongly depends on surface temperature (**T**) and energy (**E**) and ion flux (ϕ) of bombarding ions

c) Impurity Deposition and Collisional Mixing

Differential Fluence: $\Delta \Phi = \Phi / N_H$ (Φ : Total fluence, N_H : Number of pseudo ions) **Surface Thickness:** $d = \sum_{i=1}^{N} \Delta x_i (N : \text{Number of layers, } \Delta x_i : i\text{-th Layer thickness})$



Collision process of a pseudo Ion :

Reflection, Implantation, Physical Sputtering

After simulation of collision process :

Areal density of *j*-th atom in i-th layer : $A_{ij} = q_j n_i \Delta x_i + \Delta N_{ij} \Delta \Phi$ (ΔN_{ij} : Change in number of *j*-th atom in *i*-th layer) *i*-th layer thickness : $\Delta x_i = \sum_{j=1}^{N_c} A_{ij} n_{0,j}^{-1}$ ($n_{0,j}$: j-th atom density) *j*-th atom constituent in *i*-th layer : $q_{ij} = A_{ij} / \sum_{k=1}^{N_c} A_{ik}$ Maximum areal density of 1th atom in *i*-th layer : $A_{i1}^{max} = [q_1^{max} / (1 - q_1^{max})) \sum_{j=2}^{N_c} A_{ij}$ Reemission $\Delta A_{i1}^{reem} = A_{i1} - A_{i1}^{max}$ Saturation $A_{i1} = A_{i1}^{max}$ 3rd ITER International Summer School: Plasma Surface Interaction, AIX EN PROVENCE, 22-26 June 2009 **d) Thermal Diffusion of Implanted Impurities**



 \star Impurity Deposition and Collisional Mixing

★ Thermal Diffusion of Deposited Impurities Diffusion $D = D_0 \exp(-Q_D / kT)$ Coefficient D_0 : Material Constant (cm²s⁻¹) Q_D : Activation Energy (eV) T: Material Temperature (K)

- Γ : Incident Ion Flux (cm⁻²s⁻¹)
- ϕ : Total Ion Fluence (cm⁻²)
- $t : (= \phi / \Gamma)$ Irradiation Time (s)
- N : Number of Pseudo lons
- $\Delta \phi : (= \phi / N)$ Differential Ion Flux (cm⁻²)
- $\Delta t : (= t / N)$ Differential Irradiation time (s)

(B) MODELLING AND INTEGRATED SIMULATION

- a) Projectile reflection and physical sputtering
- b) Chemical sputtering and hydrocarbon emission
- c) Impurity deposition and collisional mixing
- d) Thermal diffusion of impurities in materials
- e) Impurity transport in near-surface plasmas
- f) Molecular dynamics simulation of particle solid interactions
- g) Particle-in-cell simulation of plasma/sheath on surfaces

Monte Carlo Modeling of Impurity Transport

Plasma



Carbon tile

Rate coefficients (cm³/s)

Rate coefficients (cm³/s)

Electron temperature (eV)



successively collides with plasma

The released C_xH_y molecule

electrons and ions.

(R.K.Janev, D.Reiter, Rep.FZ-Juelich, Jul-3966(2002); Jul-4005 (2003))

10⁻⁶ **10**^{-€} + C H CH е coefficients (cm³/s) C_H 10⁻⁷ 10⁻⁷ + H + 2e10⁻⁸ 10⁻⁶ 10⁻⁹ 10⁻⁹ **10⁻¹⁰** 10⁻¹⁰ Rate 10⁻¹¹ 10⁻¹¹ 10 100 1 10 100 1 Electron temperature (eV) Electron temperature (eV) **10**⁻⁶ **10**^{-€} C_H е + C.H e coefficients (cm³/s) 10⁻⁷ 10⁻⁷ $C_{2}H^{+} + H$ 10⁻⁸ 10⁻⁸ 10⁻⁹ 10⁻⁹ **10**⁻¹⁰ **10**⁻¹⁰ Rate 10⁻¹¹ + CH, 10⁻¹¹ 10 100 100 1 10 1

Electron temperature (eV)

The elastic collisions with the residual neutral hydrogen atoms

T. Motohiro, Y. Taga, Thin Solid Films, 112 (1984) 161.

The model includes

- Lorenz force $F_z = q(v \times B)$
- <u>friction force and</u>

temperature gradient thermal force

: P.C.Stangeby, *The Plasma Boundary of Magnetic Fusion Devices* (IOP, Bristol, 2000) p.296.



Debye sheath and magnetic pre-sheath potential

 ϕ_0 : sheath potential

: J.N.Brooks, Phys. Fluids B2(1990)1858.

<u>Cross-field diffusion</u>

 $D_{\perp} = 1 [m^2/s]$: K. Shimizu, T. Takizuka purakakugakkaishi 71 (1995) 1135.

Hydrocarbon Redeposition on PFW Surfaces



¹³CH₄ injection experiments at TEXTOR

roof-like test limiter exposed to SOL plasma of TEXTOR



Top of the limiter was positioned at *LCFS*, the radial position of which is r=46 cm.

At LCFS, $T_e = 54 \text{ eV}$, $T_i = 1.5T_e$ and $n_e = 1.9 \times 10^{12} \text{ cm}^{-3}$.

Radial decay of the plasma parameter: $I_{Te}=I_{Ti}=40$ mm, and $I_{ne}=22$ mm

 $^{13}\mathrm{CH}_4$ was injected into the plasma through a hole in the limiter surface.

¹²C concentration of the background plasma was taken to be 3%. (Assumption)

Most unexpected observation was the very low local deposition of ¹³C on the limiter surface (~0.2%).

Impurity transport, erosion and deposition process in EDDY and ERO codes were compared to be benchmarked against the experiments.

Observed 2D patterns of ¹³C deposition *on* the surface



Dynamics of ¹³C deposition efficiency (EDDY)

Deposition efficiency strongly changes with injection time.

With increasing injection time, deposition efficiency strongly decreases due to increasing re-erosion of the redeposited ¹³C. With further increasing time, deposition efficiency approaches a steady state value due to smaller change in the ¹³C concentration.



(B) MODELLING AND INTEGRATED SIMULATION

- a) Projectile reflection and physical sputtering
- b) Chemical sputtering and hydrocarbon emission
- c) Impurity deposition and collisional mixing
- d) Thermal diffusion of impurities in materials
- e) Impurity transport in near-surface plasmas
- f) Molecular dynamics simulation of particle solid interactions
- g) Particle-in-cell simulation of plasma/sheath on surfaces

Classical molecular dynamics (MD) simulation

The force on each atom is calculated from the analytical derivation of the interaction potential.



Integrating equation of motions of constituent atoms

Verlet algorithm :
$$\mathbf{r}_{j}(t + \Delta t) = \mathbf{r}_{j}(t) + \Delta t \dot{\mathbf{r}}_{j}(t) + \frac{\Delta t^{2} \mathbf{F}_{j}(t)}{2m}$$

 $\dot{\mathbf{r}}_{k}(t + \Delta t) = \dot{\mathbf{r}}_{k}(t) + \frac{\Delta t}{2m} \left\{ \mathbf{F}_{k}(t + \Delta t) + \mathbf{F}_{k}(t) \right\}$

Verlet algorithm is simple and high accuracy.

Coupling to an external bath (Langevin equation)

$$\lambda = \sqrt{1 + \frac{\Delta t}{\tau_T} \left(\frac{T_0}{T} - 1\right)}$$

- Δt : time step, τ_T : time constant
- T: temperature of the system, T_0 : fixed reference

temperature

It represents a proportional scaling of the velocities per time step.

Periodic boundary condition

The simulation cell is replicated throughout the space to form an infinite lattice.

If an atom leaves the simulation cell, one of its images will enter through the opposite side.

Simulation cell : should be large



More realistic, but time-consuming.



Surface Erosion and Carbon Deposition on Tungsten



22

Preparation of Realistic PFW Surfaces



- At low plasma temperature, the W is covered by deposited C and at higher temperature W-C mixed layer is formed.
- The **a-C:H layer** with different H/C is formed after a-C is bombarded with H atom.

Reflection coefficient and reflected species



Most of incident methane reflect at thermal energy and sticks at higher energy (>10eV).

High hydrogen content in amorphous carbon increases the reflection coefficients.

The W surface more increases the reflection coefficient and there reflected much more C atoms.

24

Sticking/reflection of hydrocarbons at PFWs

Incident species dependence of reflection coefficient

Incident at $CH_v(y=0\sim 4)$

Incident at $C_2H_y(y=0\sim 6)$





(B) MODELING AND INTEGRATED SIMULATION

- a) Projectile reflection and physical sputtering
- b) Chemical sputtering and hydrocarbon emission
- c) Impurity deposition and collisional mixing
- d) Thermal diffusion of impurities in materials
- e) Impurity transport in near-surface plasmas
- f) Molecular dynamics simulation of particle solid interactions
- g) Particle-in-cell simulation of plasma/sheath on surfaces

Particle-in-Cell (PIC) Simulation of Plasma and sheath



- PIC code solves the equations of motion and Poisson's equation self-consistently.
- The plasma particles with Maxwellian velocity distribution are generated at the edge region. 27
- The sheath potential vary with the charging of the wall.

Sheath Potential Profiles with Oblique Magnetic Field



The magnetic presheath is formed due to the polarization between ions and electrons.
 When the magnetic field is almost parallel to the surface, the width of MP increases.
 The heavier hydrogen isotopes have larger Larmor radius, the width of MP increases.

Energy and angular distributions of ions incident on PFW



2D-PIC simulation of plasma penetrating into Gaps



30

Plasma and sheath profiles in the toroidal gap



plasma particles can penetrate into a wide gap of 1mm.

H ion with gyro radius of 0.1mm cannot penetrate into a narrow gap of 0.2mm.

When the gap width is 0.5mm, H ion cannot deeply penetrate due to E x B drift.

31

Penetration depth of hydrocarbons in the toroidal gap

Deposition profiles of hydrocarbons at the gap sides

Effect of

10⁻¹ Toroidal gap $n = 10^{19} m^{-3}$ Redeposition rate reflection = (MD data) T =30 eV 10^{-4} 5 Ω 10 15 20 25 30 distance from tile surface (mm)

Species dependence

Using sticking coefficient calculated by MD, low energy hydrocarbons are reflected repeatedly.

The neutral species are liberated from a magnetic constrain, they are redeposited deeply.

Since the ionized particles are confined by the magnetic field and have high sticking coefficient due to sheath acceleration, they are redeposited in the gap edge. 3rd ITER International Summer School: Plasma Surface Interaction, AIX EN PROVENCE, 22-26 June 2009 Carbon Deposition in the gaps of castellated tiles

(Castellation tile used in TEXTOR)

Experiments with an ITER-like castellation geometry in TEXTOR, are studying the fuel retention and impurity transport in gaps (Litnovsky et al.).

■ In order to compare between the experiment and the simulation, we performed a simulation study of the plasma penetration and transport of hydrocarbon.

■ The plasma hydrogen ions with the maxwellian velocity distribution move along the magnetic field lines with gyration and bombard the model structure. A CH₄ molecule is released from the point bombarded by hydrogen ion.

The tile surface is inclined for varying the shadowed area.

Comparison between experimental and simulated results

We compared between TEXTOR experiment and simulation.

carbon sputtered physically are emitted from surface.

Litnovsky et al.

The redeposition layer is re-eroded by the bombardment of background plasma, therefore, C deposition is reduced at the gap edge of the rectangular cell.

Conclusions

- (I) "Erosion/deposition" on plasma facing walls in fusion devices is a critical issue related to
 - (a) global transport of impurities in plasma boundary,
 - (b) lifetime of plasma-facing components and
 - (c) tritium retention in plasma-facing components.
- (II) Modelling codes of "erosion/deposition" require to treat selfconsistently:
 - (a) Physical and chemical erosion of surface,
 - (b) Transport of released impurities above surface,
 - (c) Redeposition of returning impurities on surface and
 - (d) Resultant material mixing *below* surface
- (III) Models and assumptions in the codes have to be evaluated in cross-code and code-experiment benchmarking, whereas reliable database of physical parameters used in codes have to be prepared.

Conclusions

(continued)

(IV) Particle-in-cell (PIC) simulation of plasma and sheath above the PFW surface should be incorporated in "erosion/deposition" codes to analyze geometry effects such as gaps in the castellated tile.

 (V) In addition to existing static and dynamic Monte Carlo codes of plasma wall interactions, classical molecular dynamics (MD) simulation is an effective tool to analyze "erosion/deposition".
 But, it requires formalism of realistic interaction potentials and high performance computing.

(VI) Integration of "erosion/deposition" codes with SOL/Divertor simulation codes is an urgent issue for understanding of plasma wall interactions in fusion devices in more realistic in-vessel geometry.

THANK YOU for your attention

This work was supported by the Grant-in-Aid for Scientific Research on Priority Areas "*Tritium Science and Technology for Fusion Reactor*" of the Ministry of Education, Culture, Science and Technology in Japan

